

## Measurement of $^{76}\text{Se}(n,2n)^{75}\text{Se}$ reaction cross section with detailed uncertainty analysis at 14 MeV neutron energy

T. S. Ganesapandy<sup>1,\*</sup>, G. T. Bholane<sup>1</sup>, S. H. Patil<sup>1</sup>, S. S. Dahiwal<sup>1</sup>, V. N. Bhoraskar<sup>1</sup>, S. D. Dhole<sup>1,†</sup>

<sup>1</sup> Department of Physics, Savitribai Phule Pune University, Pune-411007, India

\* email: theosel92@gmail.com

† email: sanjay@physics.unipune.ac.in

### Introduction

Selenium (Se) is a grey non-metal found to occur in nature as seven stable isotopes. The isotopic abundance for  $^{76}\text{Se}$  is  $9.37 \pm 0.29\%$  in a natural Se sample.  $^{75}\text{Se}$  decays by electron capture with a half-life  $119.78 \pm 0.05$  days. Weeks et al. [1] explored the use of  $^{75}\text{Se}$  as a potential gamma source for brachytherapy. In literature, only a few previous works for  $^{76}\text{Se}(n,2n)^{75}\text{Se}$  reaction cross section have been reported in the experimental exchange format (EXFOR) database [2]. Only two of the reported experimental measurements were carried out with a high purity germanium (HPGe) detector and the sources of error in the experimental parameters were not adequately reported. In light of the various differences arising in the previous reported data, the re-measurement of  $^{76}\text{Se}(n,2n)^{75}\text{Se}$  reaction cross section has been carried out in the present work.

### Experimental details

Samples of Se (99.99 %) powder of natural isotopic abundance were prepared by packing 2 gm sample in polythene vials. The samples were sandwiched between two aluminium (Al) foils totally 0.285 gm and the sample size was 10 mm  $\times$  10 mm with a thickness 1 mm. The samples were irradiated at the 14MeV Neutron Generator facility [3], Department of Physics, Savitribai Phule Pune University, Pune, India. The 14 MeV neutron beam was produced by bombarding  $150 \pm 1$  keV  $\text{D}^+$  ions on an 8 Ci TiT (titanium-tritide) solid target. The neutron energy was  $14.77 \pm 0.17$  MeV determined by kinematics at the sample position. The neutron flux during irradiation was found to be  $10^8$  n/cm<sup>2</sup>/s with Al foil activation. After irradiation for an hour, the samples were transferred to the counting room. After a sufficient cooling time, the induced gamma activity of the irradiated samples was measured

with a pre-calibrated lead-shielded Ortec HPGe detector having 30% relative efficiency and 1.5 keV energy resolution at 1.33 MeV gamma energy. The decay details used in the data analysis for the present work are provided in Table 1. The irradiation time, cooling time and counting time for the present work are 3690 secs, 83268 secs and 3616 secs respectively.

**Table 1:** Decay details for sample and reference monitor reactions.

Product Nuclei	Half-life	E $\gamma$ (keV)	I $\gamma$ (%)
$^{75}\text{Se}$	$119.78 \pm 0.05$ d	136	$58.5 \pm 0.4$
$^{24}\text{Na}$	$14.997 \pm 0.12$ hr	1368.626	$99.9936 \pm 0.0015$

### Data analysis

The cross section was determined using the neutron activation equation

$$\sigma_s = \sigma_{Al} \frac{\epsilon_{Al} C_s a_{Al} A_s M_{Al} I_{\gamma Al} f_{Al} F_s}{\epsilon_s C_{Al} a_s A_{Al} M_s I_{\gamma s} f_s F_{Al}} \quad (1)$$

here, the sample reaction parameters and the monitor reaction parameters are referred to with the subscript *s* and *Al* respectively.  $\epsilon$  is the detector efficiency, *C* is the photo peak counts, *a* is the isotopic abundance, *A* is the atomic mass, *M* is the mass, *I $\gamma$*  is the branching ratio of  $\gamma$ -ray taken from Ref. [4] and *f* is the timing factor. The timing factor *f* is given by

$$f = \frac{(1 - e^{-\lambda t_1})(e^{-\lambda t_2})(1 - e^{-\lambda t_3})}{\lambda} \quad (2)$$

where,  $\lambda$  is the decay constant,  $t_1$  is the irradiation time,  $t_2$  is the cooling time and  $t_3$  is the counting time. The correction factor (*F*) due to the coincidence summing effects (*f<sub>c</sub>*) and the gamma ray self-attenuation (*f<sub>a</sub>*) is given by  $F = f_c \times f_a$ . The information related to the HPGe detector calibration, detection efficiency curve and

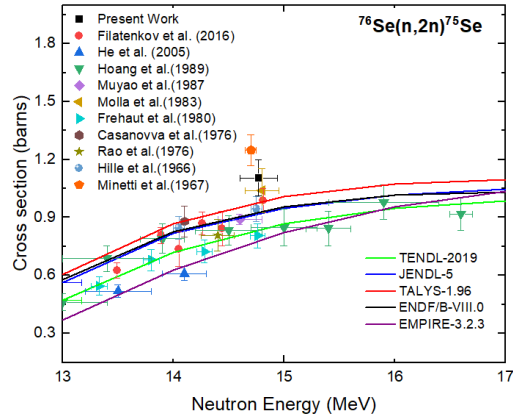
uncertainty, coincidence summing effects and self-attenuation can be found in our previous work [5]. The uncertainty in the detector efficiencies was reduced by adopting  $\eta_{Al,s} = \frac{\epsilon_{Al}}{\epsilon_s}$  where  $\eta_{Al,x}$  is the detection efficiency ratio of monitor product ( $\epsilon_{Al}$ ) to the sample product ( $\epsilon_s$ ). The details about total uncertainty propagation can be found in Section 3.3 of Ref. [6]. The partial uncertainties in the parameters of equation 1 is listed in Table 2. The total error (%) is the square root of the quadrature sum of the fractional uncertainties (%).

**Table 2:** The fractional uncertainties (%) in different parameters.

parameter	fractional uncertainty (%)
$C_s$	7.4236
$C_{Al}$	0.8967
$I_{ys}$	0.6838
$I_{yAl}$	0.0015
$\eta_{Al,s}$	1.3771
$f_{\lambda s}$	0.0415
$f_{\lambda Al}$	0.0092
$M_s$	0.0247
$M_{Al}$	0.05
$a_s$	3.0949
$\sigma_{Al}$	2.4443
<b>Total</b>	<b>8.4816</b>

### Results and Discussions

Fig. 1 shows the present result with normalized literature data [2], default statistical model calculations of TALYS-1.96 [7] and EMPIRE [8] nuclear codes. The measured cross section of  $^{76}\text{Se}(n,2n)^{75}\text{Se}$  reaction was found to be  $1.1055 \pm 0.0938$  barns at  $14.77 \pm 0.17$  MeV neutron energy. The present result is in good agreement with the data reported in Refs. [9,10] within experimental uncertainty and TALYS-1.96 default calculations. The present study lists the various sources of uncertainty and the total uncertainty in the present measurement is 8.5%. The details about uncertainty quantification will be presented during the conference.



**Fig. 1** Experimental result for  $^{76}\text{Se}(n,2n)^{75}\text{Se}$  reaction.

### Acknowledgements

One of the authors (S.D. Dhole) would like to thank SERB-DST, New Delhi sanction number: EMR/2017/002497 for the financial assistance for research work in the field of neutron induced nuclear reactions.

### References

- [1] K. J. Weeks and R.J. Schulz, *Med. Phys.*, **13**, 728-731 (1986).
- [2] N. Otuka *et al.*, *Nucl. Data Sheets*, **120**, 272:276 (2014).
- [3] V. N. Bhoraskar, *Indian J. Pure Appl. Phys.*, **27**, 648:655 (1989).
- [4] NuDat3.0, <https://www.nndc.bnl.gov/nudat3/>
- [5] T. S. Ganesapandy *et al.*, *Nucl. Phys. A*, **1023**, 122445 (2022).
- [6] T. S. Ganesapandy *et al.*, *Eur. Phys. J. Plus*, 137:711, (2022).
- [7] A. J. Koning *et al.*, *EDP Sciences*, 211:214, (2008).
- [8] M. Herman *et al.*, *Nucl. Data Sheets*, **108**, 2655:2715 (2007).
- [9] A. A. Filatenkov, IAEA Report No: INDC(CCP)-0459 (2016).
- [10] N. I. Molla, IAEA Report No: INDC(BAN)-002 (1983).